



# **ACR Workshop -Core Design & Reactor Physics-**

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**Presented to US Nuclear Regulatory Commission  
Office of Nuclear Reactor Regulation**

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# Outline

- **Overview of ACR Characteristics**
- **Major differences between current CANDU and ACR**
  - Coolant
  - Fuel
  - Lattice Pitch
- **Core Physics of ACR**
  - Negative Coolant Void Reactivity
  - Negative Power Feedback Reactivity Coefficients
  - Enhanced Control & Safety Characteristics
  - Stable Operation at All Power Levels
- **Summary**



## Reactivity Effects in ACR-700

Parameter	Value
Moderator Temperature (including density) effect	-0.013 (mk/°F )
Coolant Temperature (including density) effect	-0.006 (mk/°F )
Fuel Temperature effect	-0.008 (mk/ °F)
Power Coefficient (95% -105% full power)	-0.07 mk/% power
Reactivity change from 0% to 100% full power	-8.0 mk
Boron effect in Moderator	-2.1 (mk/ppm)
Full-Core Coolant-Void Reactivity	-3.0 mk



# ACR –700 Core Characteristics

Parameter	Value
Number of fuel channels	284
Reactor thermal power output	1982 MW
Gross Electrical Power output	731 MW
Lattice Pitch (square)	22 cm (8.7 inches)
Coolant	H <sub>2</sub> O @ 300 °C ( 572 °F )
Moderator	D <sub>2</sub> O @ 80 °C ( 176 °F )
Enrichment of CANFLEX SEU Fuel	2.0%SEU (42 pins)+ NU/Dy
Core-Average Fuel Burnup	20.5 MWd/kg(U)
Max. Fuel Element burnup	26 MWd/kg(U)
Fuel Bundles Required per FPD	5.8
Channel Visits per FPD (2-bundle-shift scheme)	2.9
Max. Time-Average Channel Power (power form factor 0.93)	7.5 MW(th)
Max. Time-Average Bundle Power	874 kW(th)
Max. Instantaneous Linear Element Rating	51 kW/m



# ACR-700 Reactor Control & Safety Systems

## Control System

- 9 Mechanical Control Assemblies with 2 segments per assembly
  - 9 mk worth for bulk- and spatial-control functions
  - 12 minutes of xenon override time
  - power cycling from 100% -75% -100%,
  - reactivity for about 7 full-power days without refueling.
- 4 Mechanical Control Absorbers for fast power reduction

## SDS1

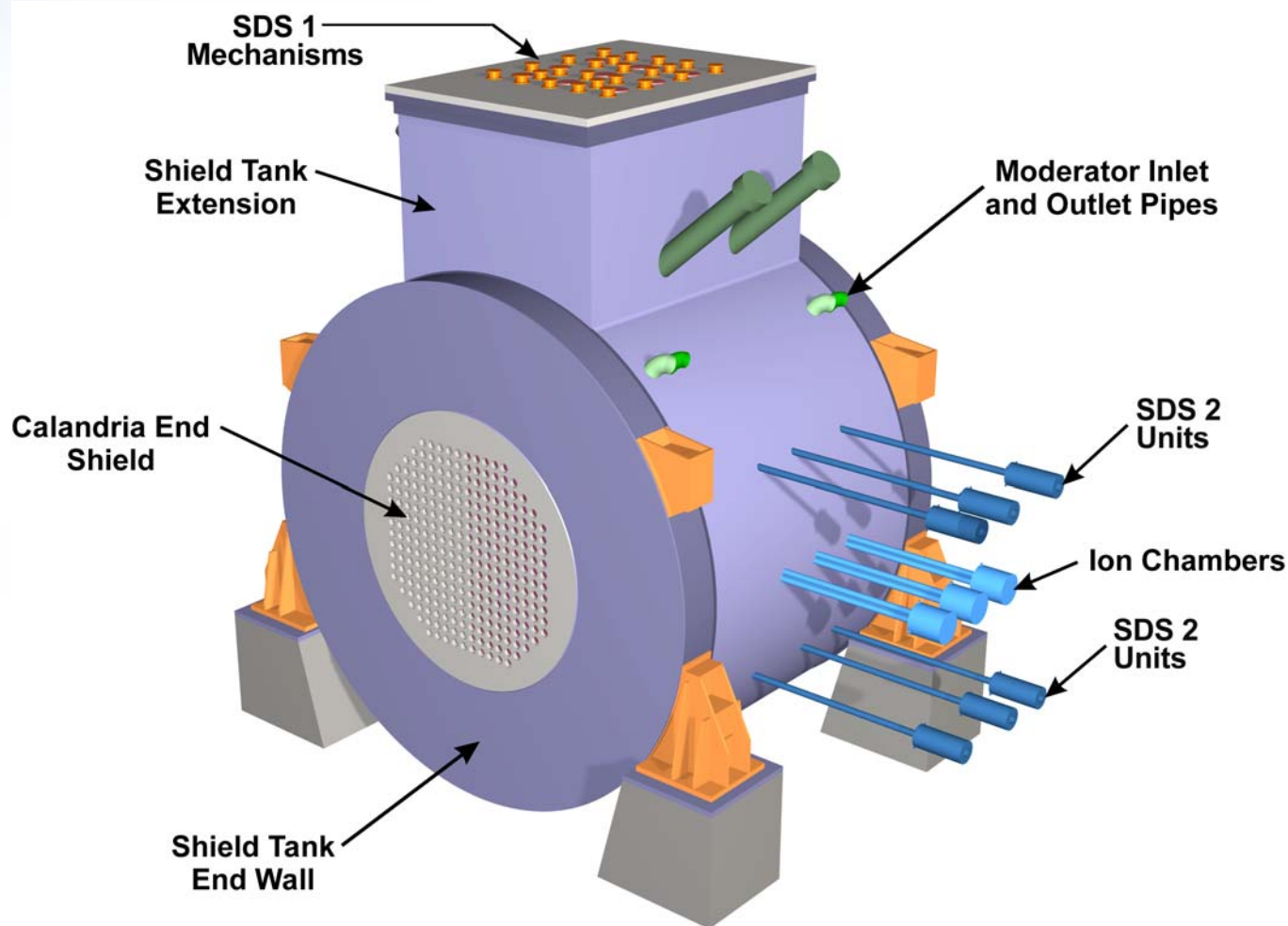
- 20 Mechanical Absorbers

## SDS2

- 6 liquid-poison injection nozzles in reflector region

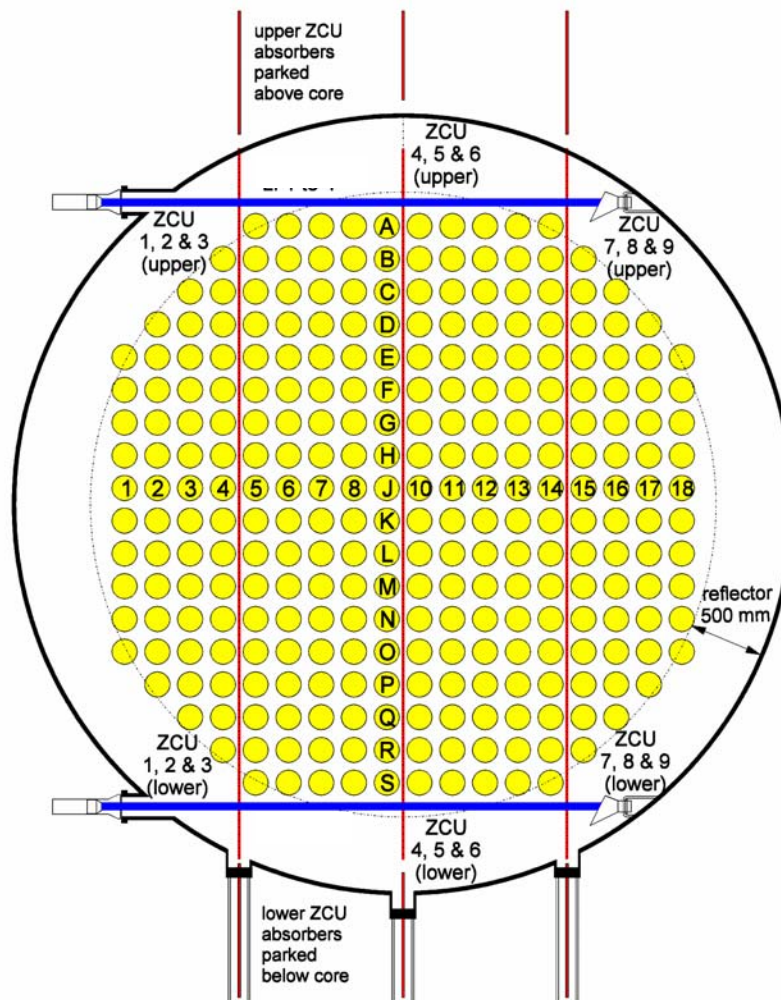


# ACR-700 Reactor Assembly





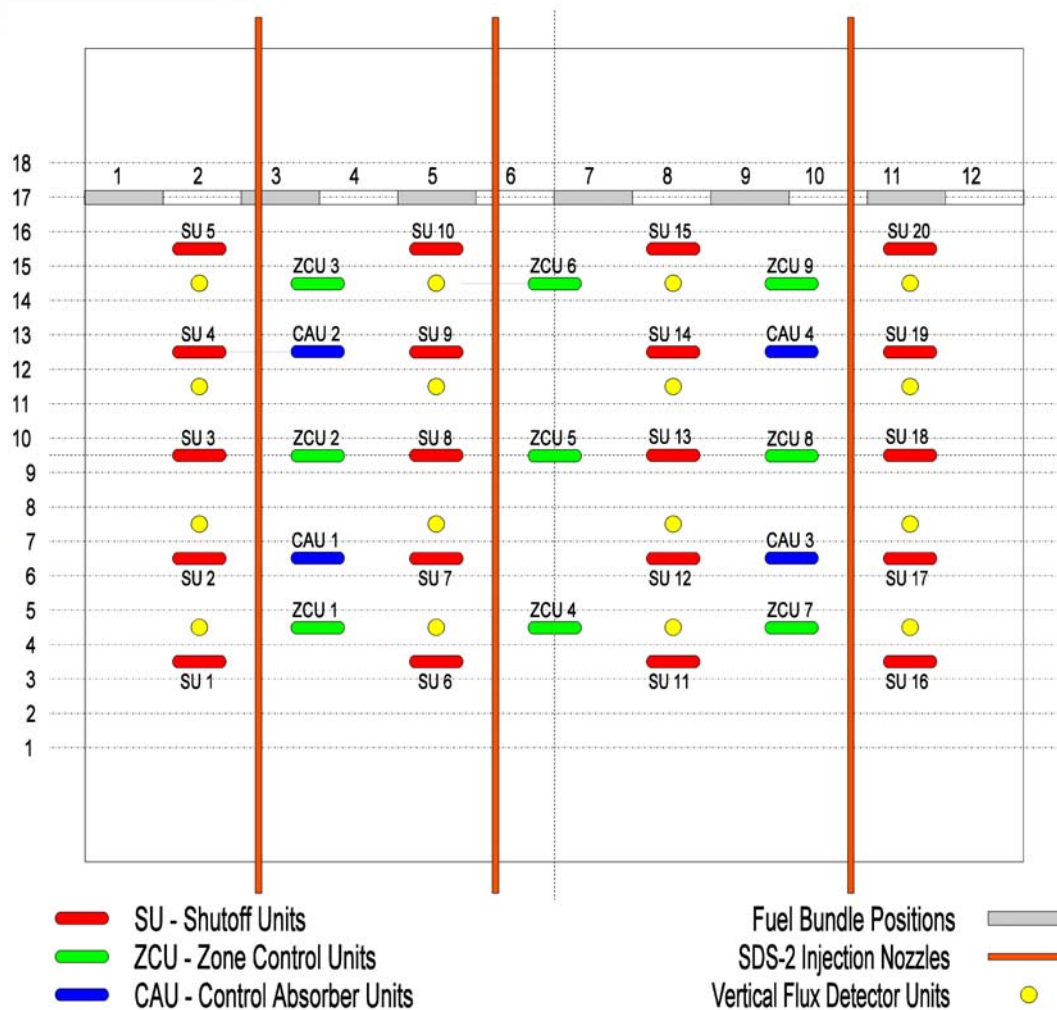
# End-View of ACR-700



ACR-700 Reactor Core  
284 Fuel Channels



# ACR-700 Reactivity Mechanisms Plan View







## Safety Parameters in ACR-700 and CANDU 6

	ACR-700	CANDU 6
Total Delayed Neutron Fraction ( $\beta$ )	0.0056	0.0058
Prompt Neutron Lifetime (millisecond)	0.33	0.92
Full-Core Coolant Void Reactivity	- 3 mk	+ 15 mk (Approx.)
Power Coefficient	-0.07 (mk per % power)	~ 0
SDS1	20 Absorber Rods	28 Absorber Rods
SDS2	6 Poison Nozzles (reflector region)	6 Poison Nozzles (core region)



## Characteristics of ACR-700 and CANDU 6

	<b>ACR-700</b>	<b>CANDU 6</b>
<b>Fuel Channels</b>	<b>284</b>	<b>380</b>
<b>Reactor Thermal Power ( MW)</b>	<b>1982</b>	<b>2064</b>
<b>Gross Electrical Power ( MW)</b>	<b>731</b>	<b>728</b>
<b>Fuel Enrichment</b>	<b>2.0% in 42 pins Central NU/Dy pin</b>	<b>37 NU pins</b>
<b>Core-Averaged Burnup (MWd/kgU)</b>	<b>20.5</b>	<b>7.5</b>
<b>Fueling Rate (Bundles per Day)</b>	<b>5.8</b>	<b>16</b>
<b>Channel Visits/Day</b>	<b>3</b>	<b>2</b>

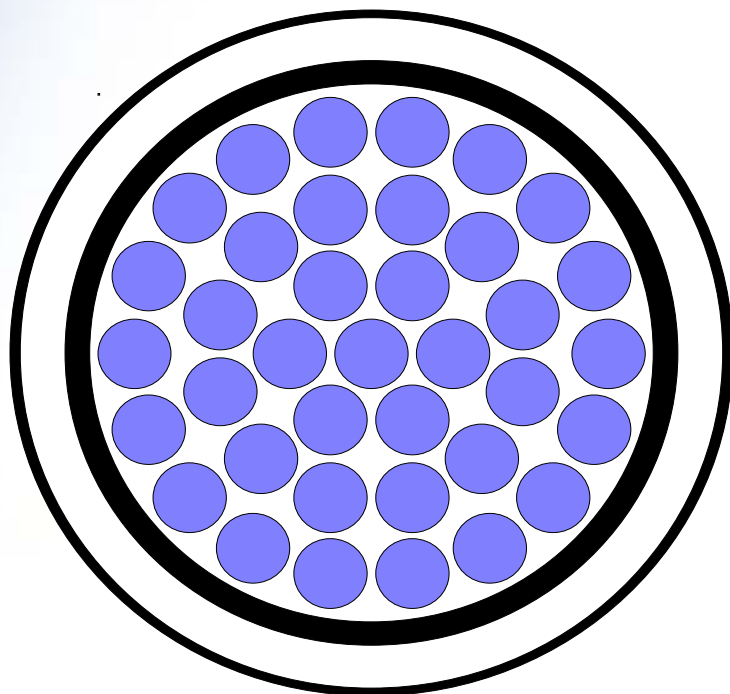


# Major Differences between CANDU 6 and ACR

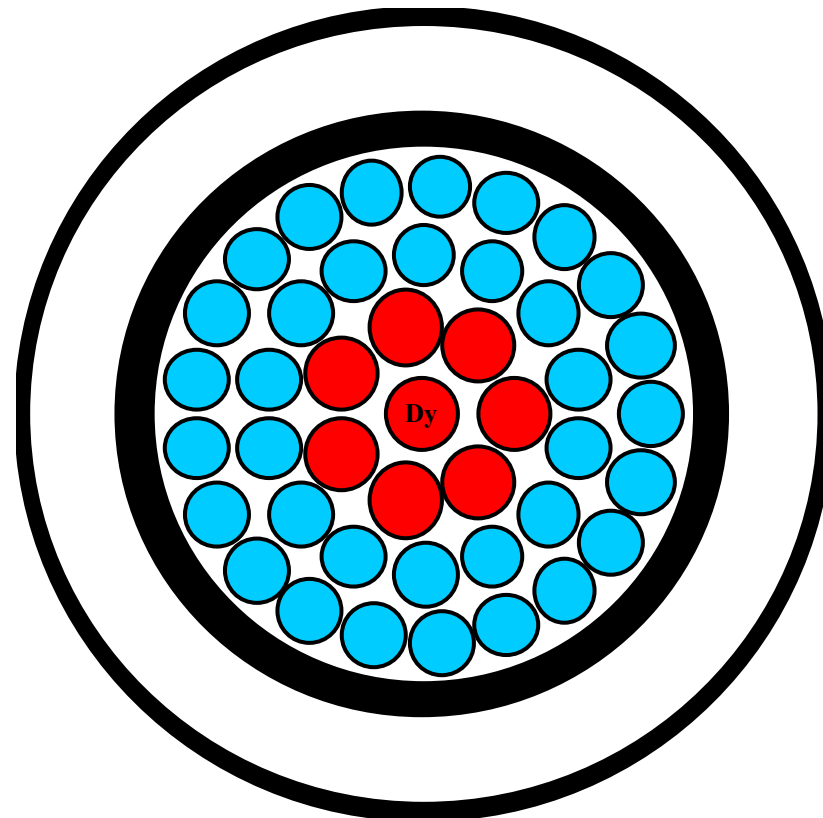
- **Coolant**
  - CANDU 6 ( $D_2O$ )
  - ACR ( $H_2O$ )
- **Fuel**
  - CANDU 6 (NU in 37-element bundle)
  - ACR (2.0 % SEU in 42 pins, Central Pin Dy/NU, CANFLEX bundle)
- **Lattice Pitch**
  - CANDU 6 (28.575 cm, 11.25 inches)
  - ACR (22.0 cm, 8.66 inches)



# CANDU Fuel Bundle Designs



*37-Element Bundle  
C6 Fuel Channel*



*CANFLEX  
Bundle (43 elements)  
ACR Fuel Channel*

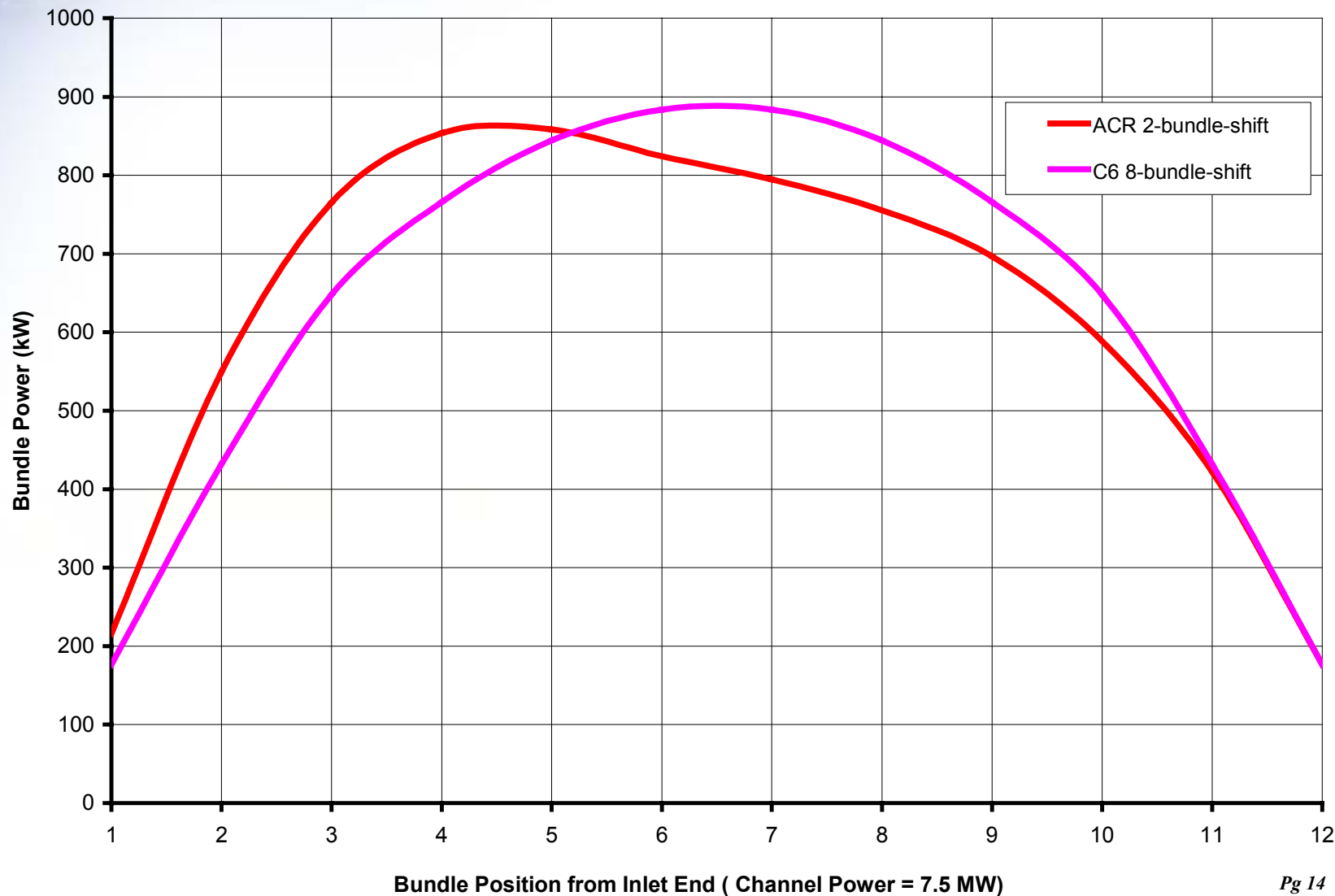


# Effects of CANFLEX SEU Fuel in ACR

- Enables the use of H<sub>2</sub>O Coolant
- Allows the reduction of moderator to reduce Coolant Void Reactivity ( CVR)
- Allows the use of neutron absorber in the central fuel pin to further reduce CVR to target of – 3 mk
- High fuel burnup
- Reduction in maximum fuel element rating
- Inlet skewed axial power profile improves thermalhydraulic margin



# Axial Power Profiles in ACR and in C6





## Effect of Coolant Void in ACR

- ACR lattice is under-moderated with normal H<sub>2</sub>O coolant
- H<sub>2</sub>O acts as both coolant and moderator
- LOCA further reduces moderation from the lattice
- Coolant Void Reactivity (CVR) is a combined effect due to loss of absorption (positive) and loss of moderation (negative) from H<sub>2</sub>O
- Increase in fast flux and decrease in thermal flux upon LOCA
- U238 and Pu239 generate negative components in CVR
  - Increase in Resonance Absorption (1 eV to 100 keV) in U238
  - Decrease in Fission (0.3 eV resonance) in Pu239



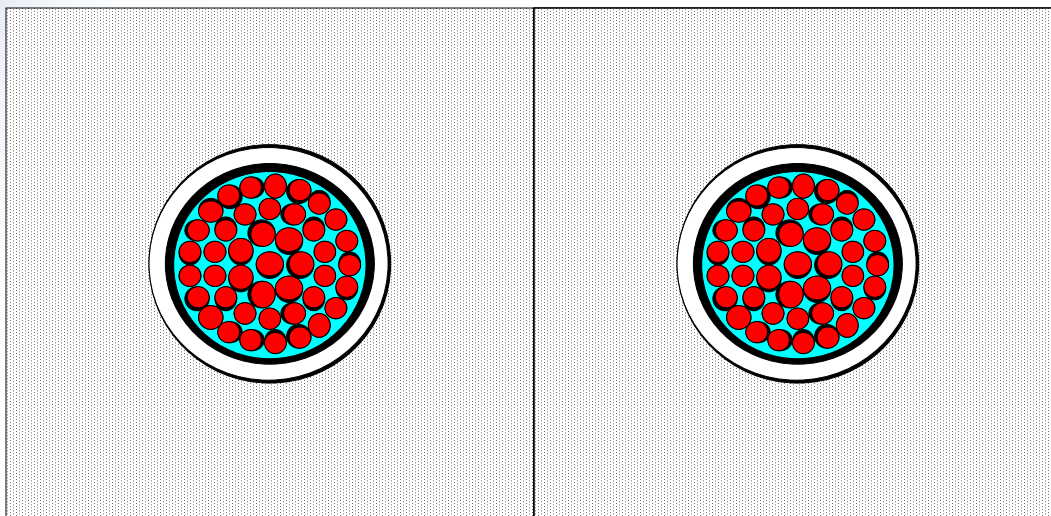
## Physics Innovations to achieve slightly negative CVR ( H<sub>2</sub>O Coolant)

- Large Moderator/Fuel ratio ( $V_m/V_f$ ) means high CVR
- Current Lattice Pitch ( LP) 28.575 cm ( 11.25 inches )  
 $V_m/V_f = 16.4$     CVR = + 60 mk
- Target CVR = -3 mk requires  $V_m/V_f < 6.0$ , 0 LP < 20 cm (7.87 inches)
- Minimum LP = 22 cm ( 8.66 inches) required to provide space for feeders between channels  
 $V_m/V_f = 8.4$
- Use larger CT, OR = 7.8 cm (3.07 inches) to displace more moderator
  - $V_m/V_f = 7.1$
  - Add Dy (4.6% ) to central NU pin    CVR = - 3 mk

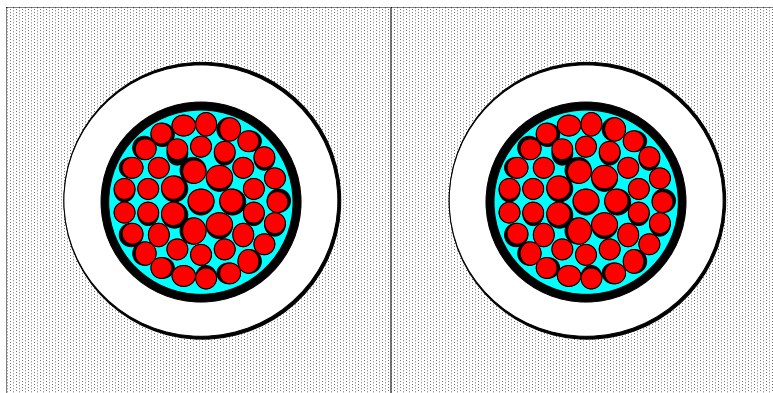




# Comparison of CANDU 6 and ACR Lattices



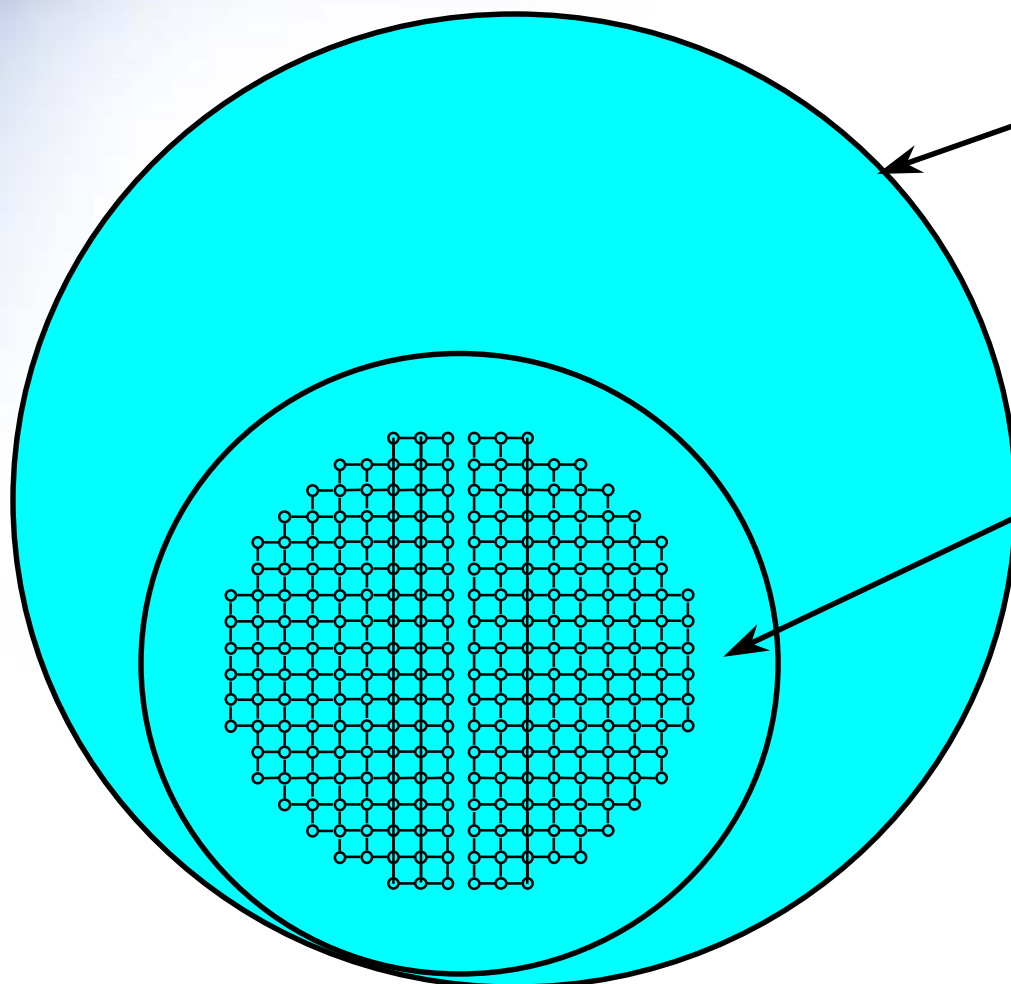
**CANDU 6 Lattice**



**ACR Lattice**



# Core Size Comparison



**CANDU 6**  
728 MWe  
380 channels  
Diameter = 760 cm  
( 299 inches)

**ACR**  
731 MWe  
284 channels  
Diameter = 520 cm  
( 205 inches)

Calandria volume reduced by a factor of 2.5 (smaller lattice pitch).

By using H<sub>2</sub>O coolant, less than 25% of D<sub>2</sub>O used in C6 is required.



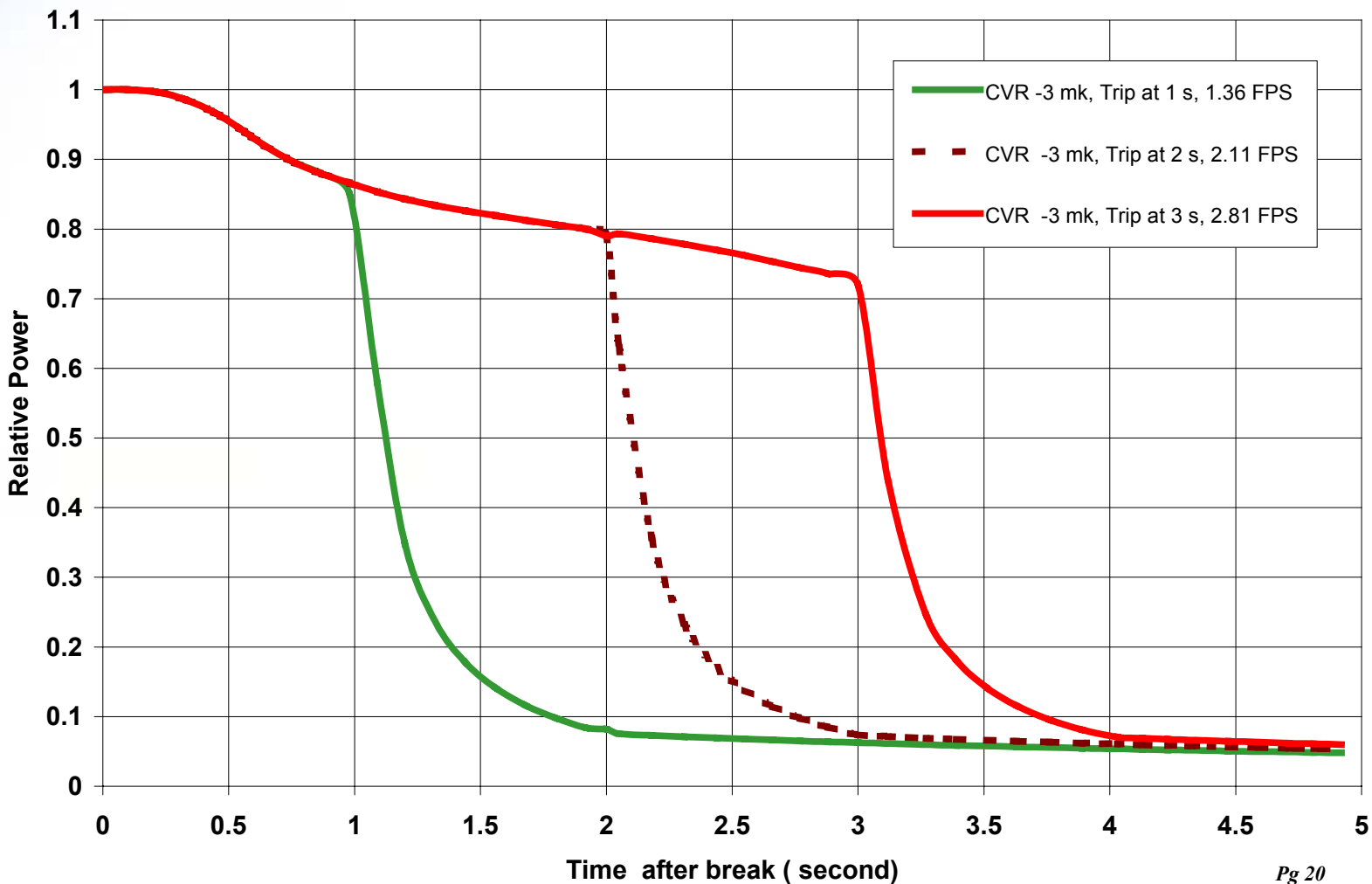
# Effect of Trip Time & CVR on LOCA Transients in ACR

- **LOCA power transients in ACR**
  - Not sensitive to trip time ( 1 to 3 seconds)
  - Not sensitive to the magnitude of the negative CVR ( -1 mk to -6 mk)



# Effect of Trip Time on LOCA Transient

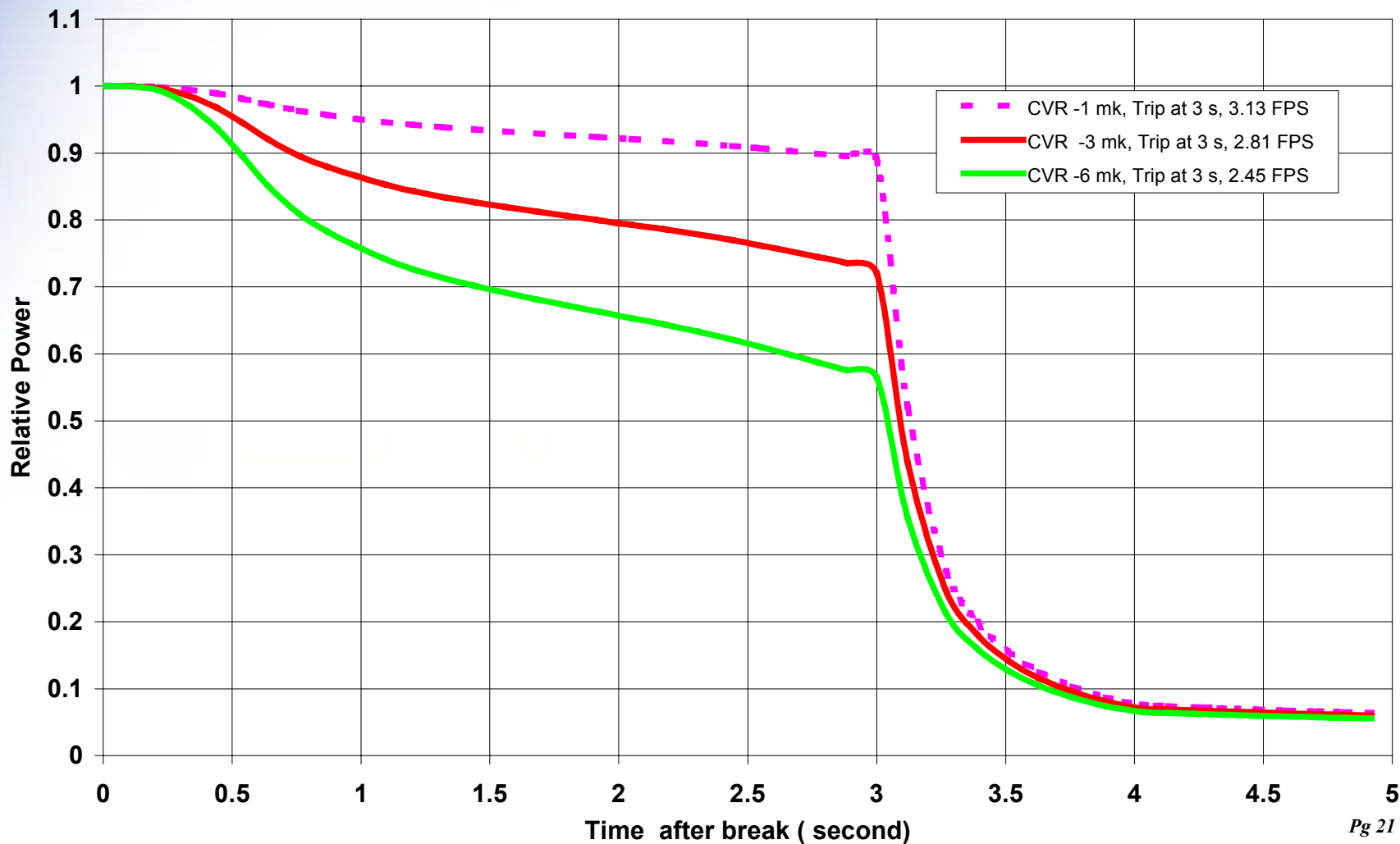
ACR 100% RIH LOCA Transient





# Effect of CVR on LOCA Transient

## ACR 100% RIH LOCA Transient



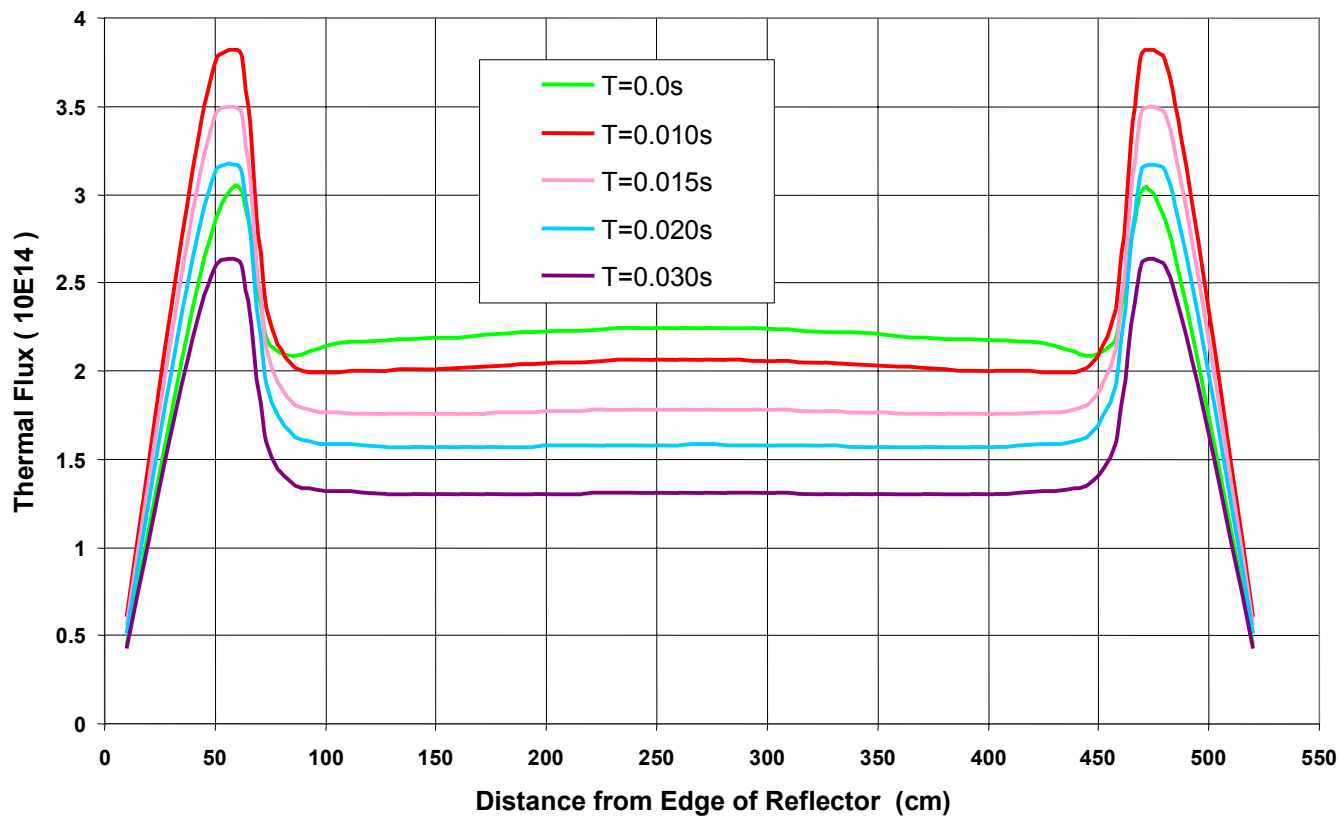


# Unique LOCA Features in ACR

- **Power in reactor core region drops upon LOCA due to negative void reactivity**
- **Rapid rise in thermal neutron flux in the reflector region due to migration and subsequent thermalization of fast neutrons from the core region**
- **Fast neutronic trip is available from neutron detectors in the reflector region**

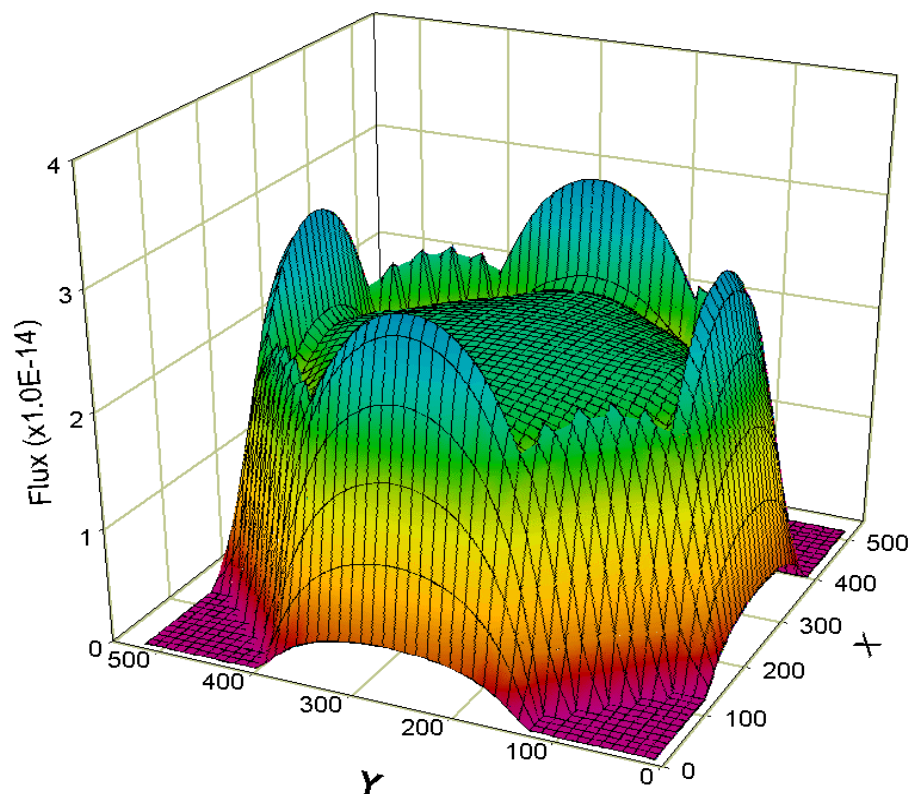
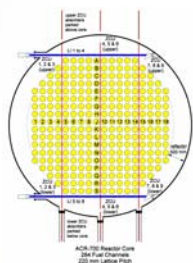


## Thermal Neutron-Flux Distributions in ACR-700 after LOCA





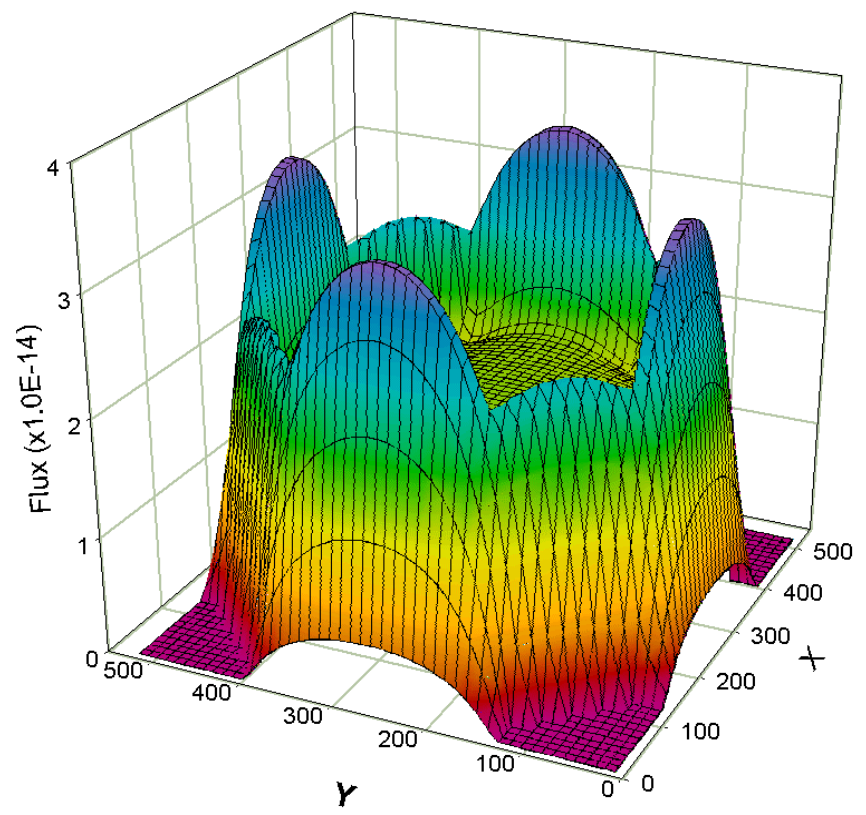
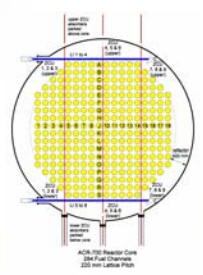
# Thermal Flux Profile upon LOCA at t=0 s





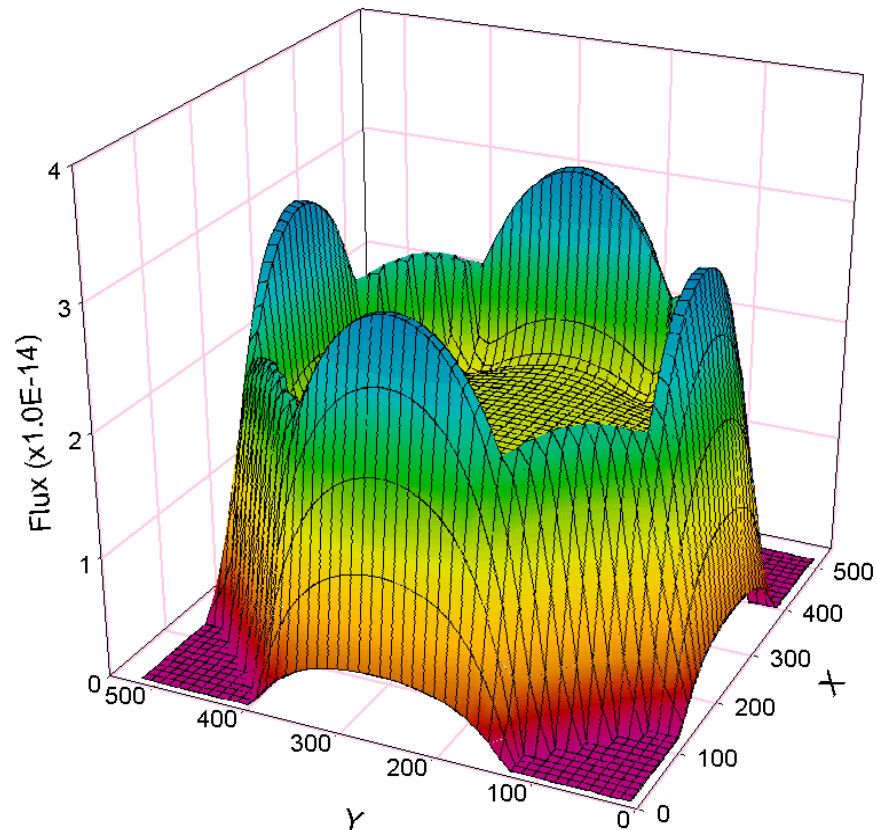
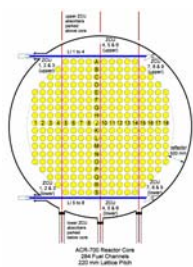


# Thermal Flux Profile upon LOCA at t=0.015 s



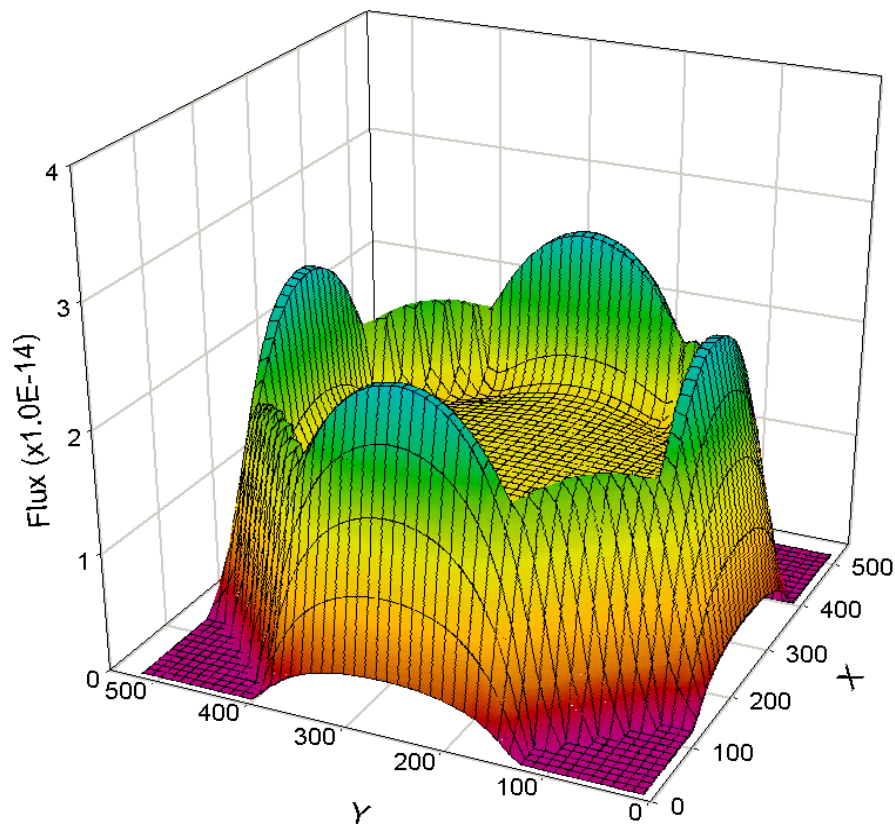
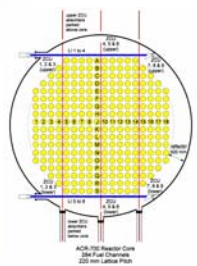


# Thermal Flux Profile upon LOCA at t=0.02 s





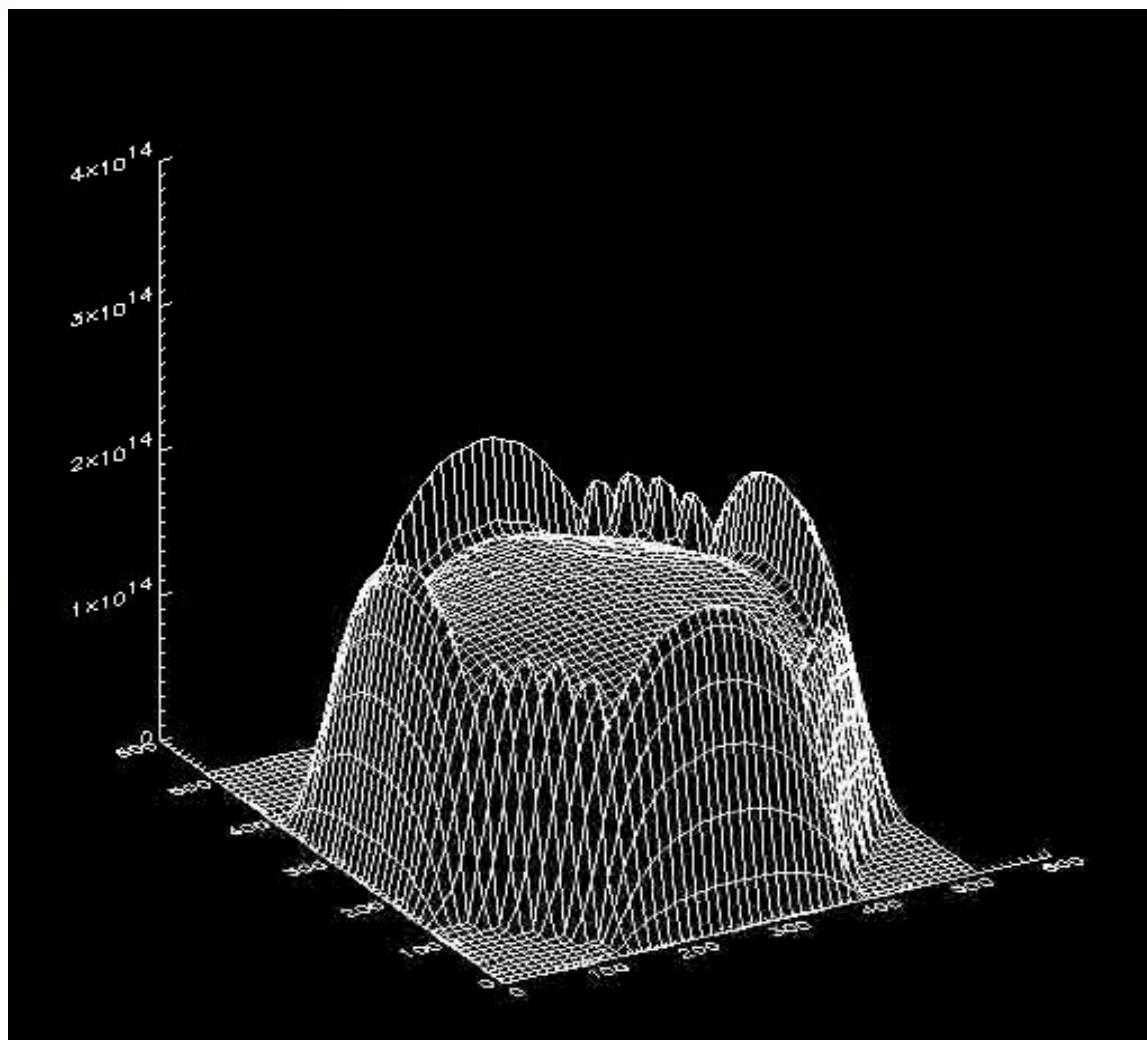
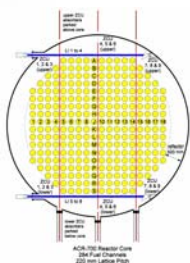
# Thermal Flux Profile upon LOCA at t=0.03 s





# Thermal Flux Profiles in ACR-700 upon LOCA

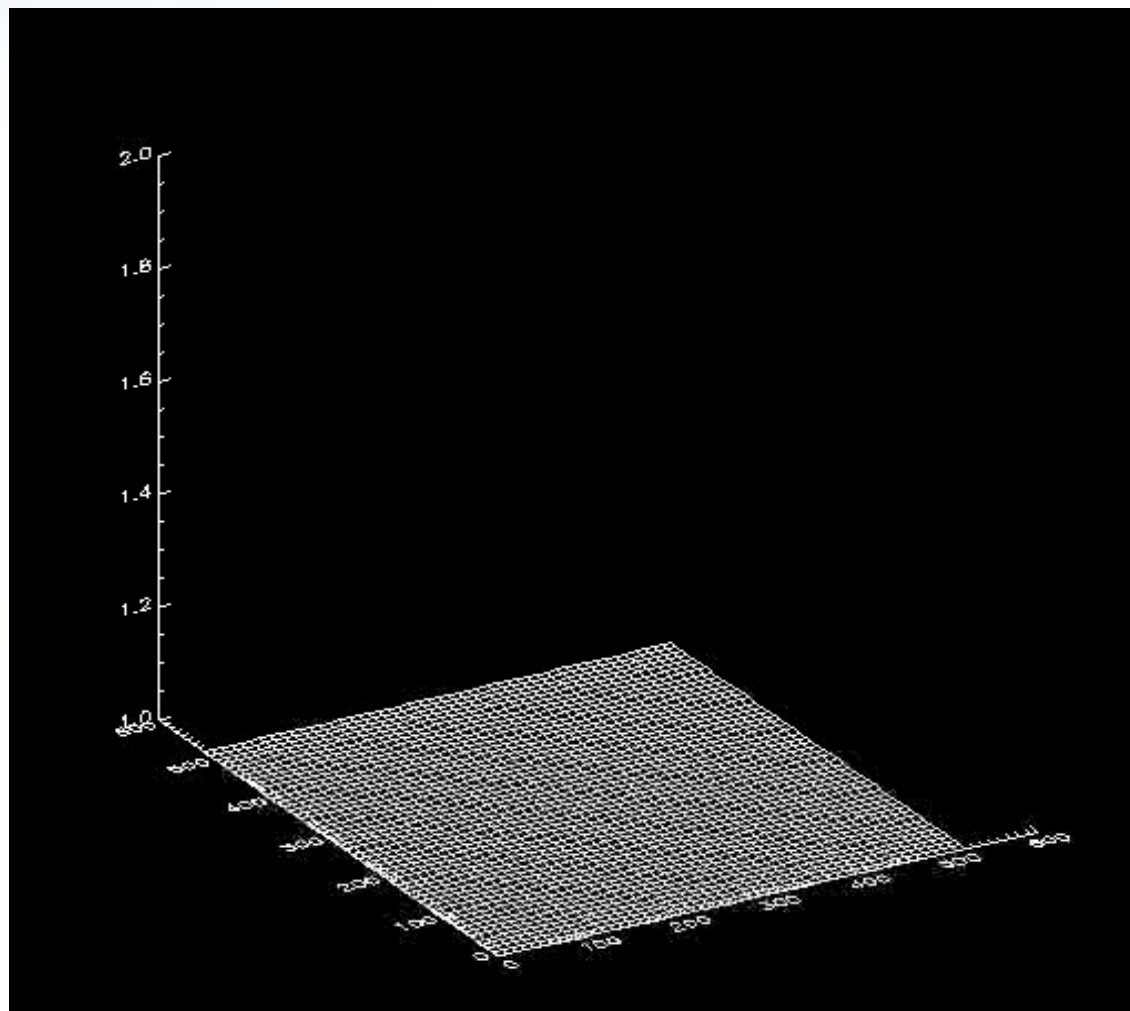
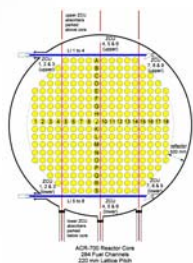
( click picture to start animation )





# Thermal Flux Ratios in ACR-700 upon LOCA

( click picture to start animation )





# Summary

- **ACR is an evolutionary design of current CANDUs**
- **Common features between ACR and current CANDUs:**
  - Horizontal fuel channels
  - D<sub>2</sub>O moderator
  - On-power fueling
  - Simple fuel bundle design
- **ACR specific features:**
  - H<sub>2</sub>O coolant
  - High burnup SEU fuel
  - Smaller lattice-pitch and compact reactor core
  - Negative coolant void reactivity enhances safety margins
  - Negative power feedback coefficients enhances reactor stability



# AECL