

1. (8 marks) A neutron and a fissile atom can interact. Describe the possible outcomes, including any nuclear reaction by-products.

2. (5 marks) Show the physical processes which are important in a reactor on a graph which has energy as the horizontal axis.

3. (7 marks) Consider the equation

$$k_{effective} = P_{FNL} P_{TNL} \epsilon p f \eta$$

Define, in words, each term.

4. (6 marks) Consider the expression

$$B_m^2 = B_g^2$$

Use mathematics and words to discuss its meaning.

5. (6 marks) Consider a parallelepiped core of dimensions  $X$ ,  $Y$  and  $Z$ . Assume uniform material properties, and that the reactor is operating at a constant power of  $P$  MW. What is the maximum flux in terms of power and reactor properties? Neglect the extrapolation length.

6. The multi-group diffusion equations may be written

$$-\nabla \cdot D_g \nabla \phi_g + \Sigma_g \phi_g = \lambda_g \sum_{g'=1}^G \nu_{g'} \Sigma_{fg'} \phi_{g'} + \sum_{g'=1}^G \Sigma_{g'/g} \phi_{g'}$$

- a. (3 marks) Develop the two-group equations for an infinite reactor.
- b. (4 marks) Use your knowledge of physical processes to simplify the equations as much as possible. Do not consider the buckling or “1½-group” approximations.

7. A flux  $\Phi_0$  n/cm<sup>2</sup>s of neutrons of energy  $E$  strikes the left side of a slab of thickness  $w$ , height  $h$  and depth  $d$ . Neutron absorption results in complete transfer of the neutron energy to the slab material. Assume (1) the beam cross-section area is larger than the slab face, (2) constant macroscopic cross-sections, and (3) no scattering. *Note: a small sketch indicating your coordinates will help all of us.*
- a. (5 marks) Calculate the linear energy deposition rate at the left and right surfaces.
- b. (4 marks) What is the total energy deposition rate in the slab?

8. (6 marks) Develop the finite-difference formulation for just the leakage term of the diffusion equation in a two-dimensional  $x$ - $y$  geometry where the diffusion coefficient  $D$  is a function of position. Assume the mesh spacing is constant but not necessarily the same in each direction.

9. (6 marks) The three-group fluxes for a bare spherical reactor of radius  $R = 50$  cm are given by the following:

$$\phi_1(r) = \frac{3 \times 10^{15}}{r} \sin\left(\frac{\pi r}{R}\right) \quad \phi_2(r) = \frac{2 \times 10^{16}}{r} \sin\left(\frac{\pi r}{R}\right) \quad \phi_3(r) = \frac{1 \times 10^{16}}{r} \sin\left(\frac{\pi r}{R}\right)$$

The group diffusion coefficients are  $D_1 = 2.2$  cm,  $D_2 = 1.7$  cm and  $D_3 = 1.05$  cm. Neglecting the extrapolation length, calculate the total leakage of neutrons at any point in the core.

Note: in spherical co-ordinates,  $\nabla^2 f = \frac{1}{r^2} \frac{\partial}{\partial r} r^2 \frac{\partial f}{\partial r}$